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Ferreirós, P.A.; Savoy Polack, E.C.; Lanzani, L.A.; Alonso, P.R.; Quirós, D.P.; Mieza, J.I.; Rubiolo, G.H.

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# Effects of thermo-mechanical process on phase transitions, hydrogen solubility and corrosion of Ta-modified Zr-1Nb alloys

P.A. Ferreirós<sup>a,b,c</sup>, E.C. Savoy Polack<sup>a</sup>, L.A. Lanzani<sup>a</sup>, P.R. Alonso<sup>a</sup>, P.D. Quirós<sup>a</sup>, J.I. Mieza<sup>a</sup>, G.H. Rubiolo<sup>a,b,\*</sup>

<sup>a</sup> Gerencia Materiales (GAEN) - Comisión Nacional de Energía Atómica (CNEA), Instituto Sabato - Universidad Nacional de San Martín (UNSAM), Av. Gral. Paz 1499, San Martín, Buenos Aires B1650KNA, Argentina.

<sup>b</sup> Consejo Nacional de Investigaciones Científicas y Técnicas (CONICET), Godoy Cruz 2290, C1425FQB Ciudad Autónoma de Buenos Aires, Argentina

<sup>c</sup> School of Metallurgy and Materials, University of Birmingham, Birmingham, B15 2TT, UK.

**Keywords:** Zr–Nb alloy, Ta addition, phase transformation, corrosion, hydrogen solubility

## Abstract

The feasibility of using Ta as a minor alloying element in Zr-1Nb alloys is addressed in terms of phase stability and resistance to corrosion of the microstructure obtained with the following two thermomechanical processes after a water quenching from  $\beta_{Zr}$ : a) annealed at 570 °C for 3840 h and b) a combination of intermediate annealing temperatures and cold rolled steps. Hydrogen solubility was also measured, but only alloys with the first process. It was found, through calorimetric measurements, that an increase of Ta replacing Nb in the nominal alloy composition increases the  $\alpha \rightarrow (\alpha + \beta)$  phase transformation temperature in agreement with an estimated increase in the enthalpy of formation of the  $\beta_{Nb/Ta}$  phase,  $\Delta H_{\beta_{Nb/Ta}}^f$ . As expected, the mass fraction of  $\beta_{Nb/Ta}$  measured in the rolled alloys is lower. Further, this diminution is larger in the alloys containing Ta due to the lower mobility of Ta and higher  $\Delta H_{\beta_{Nb/Ta}}^f$ . The growth of the oxide layer in each alloy, at 400 °C steam up to 14 days, follows the predictor calculated as the ratio between the atomic concentrations of Nb plus Ta in the alloy and  $\Delta H_{\beta_{Nb/Ta}}^f$ . A slight decrease in hydrogen solubility was observed at low Ta content in the alloy.

## 1. Introduction

The Zr-1Nb alloy is being used as an advanced cladding material for French pressurized water reactors (PWR), under the designation M5, and for similar application in Russian reactors under the designation E110. The absence of Sn in their chemical composition significantly improve corrosion and hydrogen-pickup resistance in high burnup [1]. Not only does the alloying-element content play a major role in their corrosion kinetics, but so too does its

\* Corresponding author. Present address: Gerencia de Materiales, Departamento de Transformaciones y Propiedades, Comisión Nacional de Energía Atómica, Av. Gral Paz 1499, B1650KNA San Martín, Argentina. Tel.: 054 11 6772 7240; fax: 054 11 6772 7362. E-mail address: rubiolo@cnea.gov.ar (G.H.Rubiolo).

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4 spatial distribution in the material. Their corrosion resistance is known to increase when  
5 equilibrium particles of  $\beta_{Nb}$  phase (very rich in Nb) are present instead of the metastable  $\beta_{Zr}$   
6 phase (19 wt% Nb) [[2],[3],[4]]. A recent investigation provided new experimental  
7 information seeking to elucidate the oxidation behavior of Nb present in  $\beta_{Zr}$ ,  $\beta_{Nb}$  and solid  
8 solution and how this is related to the macroscopic oxidation kinetics of Zr-Nb alloys [5]. The  
9 authors have shown that  $\beta_{Nb}$  precipitates are much more resistant to oxidation than  $\beta_{Zr}$   
10 precipitates, suggesting that niobium-rich precipitates exhibit a protective effect on  
11 immediate oxidation of inner niobium, allows it to remain metallic well after oxidation of the  
12 surrounding zirconium matrix. Consequently, they also found a larger doping of the oxide  
13 with Nb in solid solution when the alloy contains  $\beta_{Zr}$  precipitates. Additionally, the corrosion  
14 kinetics of  $\beta_{Zr}$  containing alloy is parabolic, whereas that containing  $\beta_{Nb}$  exhibits  
15 subparabolic kinetics. Using their own model of the oxidation kinetics of zirconium alloys [6],  
16 which includes space charge effects within the oxide, they theorized that this additional Nb  
17 in the solid solution in the oxide is able to compensate for the space charge. Thus, they can  
18 correlate the parabolicity of the oxidation kinetics with the aliovalent ions dispersed in  
19 solution in the oxide; the lower the concentration of aliovalent ions in solution, the lower  
20 the spatial charge compensation and the subparabolic oxidation kinetics occurs.  
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30 Mo and Ta are two of the promising minor alloying elements for doping in Zr-Nb alloys. Both  
31 systems, Nb-Mo [7] and Nb-Ta [8], are complete solid solution systems, due to almost  
32 identical atomic radii and the same crystal structure with similar lattice parameters. The  
33 binary phase diagrams, Zr-Nb [9] and Zr-Ta [10], have similar phase fields and monotectoid  
34 reactions; instead, the Zr-Mo [11] phase diagram reports the formation of an intermetallic  
35 compound,  $Mo_2Zr$ , which can result in the formation of a new type of precipitates beyond  
36 the  $\beta_{Nb}$  phase. However, viewed from the material selection philosophy for nuclear fuel  
37 cladding, the thermal neutron absorption cross section of Mo (2.48 barns) is quite low, while  
38 that of Ta (20.6 barns) is too high [12]. Still, the neutron economics of replacing Nb with Ta  
39 in the Zr-1Nb alloy for use in the cladding material must be evaluated. A first estimate of the  
40 effect can be made by calculating the relative change in the macroscopic thermal neutron  
41 absorption cross section of the alloy [13]. Thus, we obtain that the macroscopic cross section  
42 increases almost linearly by substituting Nb for Ta and reaches a maximum of 19% for the  
43 total replacement. This means that the neutron economy will not change if the thickness of  
44 the cladding decreases following the same rate, but in that case the circumferential stress  
45 will increase in the same way. The effects of Mo on the mechanical properties of Zr-Nb alloys  
46 were investigated by Yang et al [14], they were able to estimate a 13% increase in the yield  
47 stress of Zr-1.2Nb caused by 0.1 wt% Mo in solid solution. Our previous investigation on the  
48 chemical composition of the precipitates and the matrix in equilibrium ( $\alpha + \beta$ ) Zr-1Nb alloys  
49 with Ta addition showed a matrix with 0.17 wt% Ta in solid solution [15]. Due to the almost  
50 identical atomic radii for Mo and Ta, solid solution hardening should only depend on atomic  
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4 concentration, therefore a yield stress increase of around 22% can be expected with Ta  
5 replacing Nb in the Zr-1Nb alloy. This means that the larger neutron absorption of the  
6 cladding material where Nb is replaced by Ta could be compensated by decreasing its  
7 thickness without detriment to its mechanical strength.  
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11 The development of Ta-modified Zr-Nb alloys as a new nuclear cladding material has been  
12 launched with our study mentioned above [15]. That research focused on the influence of a  
13 small addition of Ta on the phase equilibrium microstructure of the Zr-1Nb alloy aged at 570  
14 °C. It was found that the microstructure of two phases ( $\alpha_{Zr} + \beta_{Nb}$ ) remains, the Ta atoms  
15 mostly replace the Zr atoms in both phases and, if the substitution of Nb for Ta maintains  
16 the composition (Nb + Ta) close to 1 wt%, a decrease in the mass fraction of the  $\beta$  phase is  
17 observed. On the account of the fact that the precipitation behavior is unambiguously  
18 influenced by the doping of Ta, profound influence will, without doubt, be exerted on that  
19 precipitation during the manufacture procedure of the fuel cladding tube. If so, the corrosion  
20 behavior of Ta-modified Zr-1Nb alloys with different manufacturing processes is expected to  
21 be different beyond the replacement of Nb by Ta. Other interesting information to know are  
22 the hydrogen absorption characteristics of these alloys, since excessive hydrogen uptake  
23 during oxidation has a deleterious effect on the ductility of cladding material. The work  
24 presented herein attempts to answer in part these questions and elucidate if Ta addition to  
25 Zr-1Nb alloy is to be favorably considered in the future alloy development.  
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## 33 34 **2. Materials and methods**

### 35 36 **2.1 Model Zr-1Nb alloys with Ta addition**

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39 The three alloys used in this study were processed from 25 g button melted in an electric arc  
40 furnace with a tungsten electrode and a water-cooled copper crucible in an argon  
41 atmosphere. The buttons were turned and remelted several times to promote chemical  
42 homogeneity. In the melting process, weight losses of less than 0.04 % were obtained.  
43 Sponge Zr ( $68 \pm 16$  ppm Hf,  $148 \pm 24$  ppm Fe,  $628 \pm 63$  ppm O,  $30 \pm 4$  ppm N and  $45 \pm 1.5$   
44 ppm H), Nb and Ta elements with purity better than 99.9 wt% were used. Table 1 shows their  
45 nominal compositions, the absolute error in the chemical composition was estimated  
46 considering the total weight loss in the melt successively assigned to each element of the  
47 alloy.  
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54 The buttons were wrapped in a tantalum foil, encapsulated into a quartz tube which was  
55 evacuated and backfilled with argon, heat treated at 1100 °C for 30 minutes and followed by  
56 a water quenching ( $\beta_{Zr}$  quenched).  
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59 The alloy buttons were sectioned parallel to their axis of revolution, using a low-speed  
60 diamond saw, to extract a coupon approximately 3 cm long and 1.5 cm wide from the central  
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part of each button for further rolling process. The two remaining parts of the button were used to reach the microstructure of precipitated phases in thermodynamic equilibrium at 570 °C.

The formation of secondary precipitate phases (SPP) from the  $\beta_{Zr}$  quenched condition has already been studied in these alloys by isothermal annealing at 570 °C up to 3840 h and was reported in reference [15]. The vast majority of the SPP in the three alloys were identified as a bcc crystal structure (called  $\beta$  phase). The chemical composition and mass fraction of the phases present are included in Table 1. The rest of the alloys material that has reached the state of thermodynamic equilibrium at 570 °C provided two solid cylinders of approximately  $\varnothing$  3.5 x 5.6 mm and a sheet of approximately 108 mm<sup>2</sup> in area and 1 mm thick for specific tests of the present research, both were extracted from them using electro-discharge machining (EDM).

**Table 1.** Chemical composition (wt%) of alloys and phases in thermodynamic equilibrium at 570 °C. Mass fraction (wt%) of both phases is also included [15].

Alloy	Phase	Chemical composition (wt%)			Mass fraction
		Zr	Nb	Ta	
Zr-1.05Nb		98.9485 ± 0.0004	1.052 ± 0.039	----	----
	$\alpha$	99.43 ± 0.06	0.57 ± 0.03	----	99.42 ± 0.09
	$\beta$	17.17 ± 1.31	82.83 ± 1.31	----	0.58 ± 0.09
Zr-0.85Nb-0.20Ta		98.9483 ± 0.0003	0.851 ± 0.032	0.201 ± 0.032	-----
	$\alpha$	99.30 ± 0.08	0.52 ± 0.03	0.17 ± 0.11	99.55 ± 0.09
	$\beta$	14.00 ± 0.84	73.86 ± 1.81	12.14 ± 2.65	0.45 ± 0.09
Zr-0.85Nb-0.40Ta		98.7510 ± 0.0004	0.850 ± 0.035	0.399 ± 0.035	-----
	$\alpha$	99.21 ± 0.09	0.52 ± 0.03	0.27 ± 0.12	99.54 ± 0.10
	$\beta$	9.91 ± 0.75	71.35 ± 2.22	18.74 ± 2.98	0.46 ± 0.10

The coupons of the zirconium model alloys were processed in the laboratory to sheet following a sequence of steps that resemble the cold pilgering and annealing to final size of zirconium-base alloys tubing material [16]. The quenched coupon was 60% hot-rolled after a pre-heating at 570 °C for 30 min, annealing 3 hours at 570 °C in vacuum and 50% cold-rolled two times to a final thickness of 0.8 mm. The 50% cold rolling step was performed with 10 passes. Between the cold rolling steps, the sheet was annealed at 570 °C for 3 h in vacuum.

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4 The final dimensions of the sheet were approximately 0.8x25x120 mm. Several test samples  
5 were removed from the rolled alloys sheet, EDM cut  $\varnothing$  3.5 mm disc type sample and 10 mm  
6 side rectangular prism type cut with diamond saw. The grain microstructure after the rolling  
7 sequence was observed with the aid of an optical microscope and polarized light. The  
8 examined surfaces were the rolling direction (RD)–normal direction (ND) plane and they  
9 were metallographically prepared using SiC papers (1200# in the final step) and then etched  
10 with a 10HF–45HNO<sub>3</sub>–45H<sub>2</sub>O (vol.%) solution. The microscope used was an Olympus BX60M.  
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## 15 2.2 Phase analysis techniques 16

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18 The final arrangement of the SPP in the microstructure of alloys was studied by light  
19 microscopy using brightfield-darkfield contrast and scanning electron microscopy (SEM)  
20 using secondary electron mode. The microscopes used were an Olympus BX60M and a FEI  
21 Quanta 200, respectively. The examined surfaces were metallographically prepared by  
22 polishing with abrasive paper up to grade 600 and etching with 10HF-45HNO<sub>3</sub>-45C<sub>3</sub>H<sub>8</sub>O<sub>3</sub>  
23 (vol.%) by swabbing for 25 seconds. In the case of the rolled alloys, the examined surfaces  
24 were the RD–transverse direction (TD) plane.  
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29 During this study, high sensitivity differential scanning calorimetry (DSC) was used as a  
30 reliable tool to quantify the global mass fraction of the  $\beta_{Nb}$  phase present in the rolled alloys  
31 [17]. For this, the material of each alloy that has reached the state of thermodynamic  
32 equilibrium at 570 °C was used as a calibration standard for the dissolution enthalpy of the  
33  $\beta_{Nb}$  phase. The transformation temperature and the dissolution enthalpy of the SPP in each  
34 thermomechanical condition of the alloy was determined from DSC measurements (Setaram  
35 LABSYS evo-DSC) with a heating rate of 5 °C/min, under an Ar (99.9997%) dynamic  
36 atmosphere of 25 ml/min. The instrument was calibrated using melting point of pure In, Sn,  
37 Al, Fe. Measurements were made in alumina crucibles that suit to the volume of a  $\varnothing$  3.5 x  
38 5.6 mm cylindrical sample. The DSC sample used for the alloys in the equilibrium state was  
39 the solid cylinder and for rolled alloys was a stack of several disks. The heat flow against  
40 temperature profile depicts the heat capacity at constant pressure of the testing sample, if  
41 an endothermic or exothermic reaction takes place, the profile shows peaks. The measured  
42 descriptors of such reactions are onset, endset, and the area under the peak calculated once  
43 the interpolated baseline of the profile is subtracted. The temperature onset of the reaction  
44 is calculated as the intersection between the baseline and a tangent line drawn on the front  
45 slope of the peak curve. Its enthalpy is calculated as the integrated area bounded by the  
46 interpolated baseline and the intersections with the peak curve. The absolute error of  
47 enthalpy was estimated by doing the same calculations with the upper and lower envelopes  
48 of the signal noise at the peak curve. When the SPP and the allotropic  $\alpha_{Zr} \leftrightarrow \beta_{Zr}$  phase  
49 transformations take place in the same range of temperatures, a deconvolution of the  
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4 different enthalpic peaks is applied, assuming that the allotropic phase transformation has a  
5 lognormal Gaussian form [18].  
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### 7 8 2.3. Hydrogen solubility measurements 9

10 Hydrogen solubility was measured only in the alloy materials that has reached the  
11 thermodynamic equilibrium state at 570 °C. The second  $\emptyset$  3.5 x 5.6 mm cylindrical sample of  
12 each alloy was subjected to the charge of hydrogen gas in a Sievert-type equipment to  
13 increase its hydrogen concentration by 200 wt ppm. Given the low mass of the sample, a  
14 high hydrogen concentration was chosen to obtain a higher mass fraction of hydrides and  
15 thus improve the DSC signal. Prior to the charging procedure, the surface of the sample was  
16 ground with 600-grit SiC abrasive paper. The incorporation of hydrogen in each sample was  
17 carried out at 550 °C under a hydrogen atmosphere (purity 99.999%) at low pressure (around  
18 10 Torr). Due to the high temperature used for charging, hydrogen fully diffuse in the  
19 characteristic lengths of the sample assuring its homogeneous distribution. As far as the  
20 previous aging process is by far longer than hydriding process, it is not expected any effect  
21 on the sample microstructure.  
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29 When the hydrogen concentration in the bulk of the alloy exceeds the terminal solid  
30 solubility (TSS), hydride precipitation occurs. It is well-known that in zirconium and its alloys  
31 the TSS does not have a unique value, but depends on the direction of approach to  
32 temperature, i.e., on whether hydrides are dissolving (TSSD) or precipitating (TSSP) [19].  
33 Among other methods, DSC allows the experimental determination of the dissolution and  
34 precipitation temperature of zirconium hydrides. A typical DSC test consist in at least three  
35 consecutive thermal cycle runs [20]. The thermal cycle is a heat up ramp from room  
36 temperature up to a maximum temperature,  $T_{max}$ , followed by a cooldown ramp to room  
37 temperature, with a hold-time of 10 min at  $T_{max}$ . The first run serves to condition the sample  
38 to a similar state before any further runs, if the responses of the next two runs are similar,  
39 i.e. the measured hydride dissolution temperatures are within 1 K, the results are considered  
40 acceptable.  
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48 The hydride transformation temperatures were measured with the same calorimeter  
49 mentioned in section 2.2. Furthermore, the crucible, the reference sample, and the  
50 atmospheric conditions were the same. The heating and cooling rates were 10 °C/*min* and  
51  $T_{max} = 500$  °C. The signals obtained from the dissolution and precipitation phenomena  
52 were processed to determine the peak temperature (PT) and the maximum slope  
53 temperature (MST), which are the most used in the literature to correlate with the TSS.  
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### 57 58 2.4. Corrosion test 59 60 61 62 63 64 65

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4 Corrosion test was performed in a 4 liter static autoclave (filled with deionized water,  $\kappa =$   
5  $1 \mu S/cm$ ) in steam at a temperature of  $(400 \pm 10)^\circ C$  and a pressure of  $(10 \pm 1)MPa$  according  
6 to the ASTM G2/G2M-06 standard test [21]. This is an accelerated test used for qualifying  
7 zirconium-based alloys for use in nuclear reactor applications. It specifies use of water with  
8 a deaeration practice to reduce dissolved oxygen to low levels. To validate the 14 days  
9 corrosion tests, three Zircaloy-4 Atucha I tube coupons (named T, M and B) with known  
10 performance were used as controls. A rack and several hooks made of Zircaloy-4 allow all  
11 the test samples to be hung inside the autoclave, the control coupons were placed at the top  
12 (T), middle (M) and bottom (B) of the enclosure while the alloy Zr-Nb-Ta samples were placed  
13 in the middle part. Prior to autoclaving test, all the specimens were polished with SiC paper  
14 down to 1200 mesh and pickled with a solution of 50 vol.% deionized water + 47 vol.%  $HNO_3$   
15 + 3 vol.% HF. After this, they were rinsed in deionized water at  $80^\circ C$  to eliminate traces of  
16 the fluoride ion coming from the pickling bath, dried with acetone and hot air and stored in  
17 a desiccator. The corrosion resistance of the specimens was evaluated by measuring their  
18 weight gain per unit area ( $\Delta w$ ) for different durations ranging from 3 to 14 days. Before and  
19 after the test, all the specimens were weighed in an analytical balance with an accuracy of  $\pm$   
20  $0.01$  mg. The Zircaloy-4 Atucha I tube coupons were measured with a caliper (accuracy  $\pm$   
21  $0.005$  mm). Due to the irregular shape of the Zr-Nb-Ta alloy samples, their areas were  
22 measured from an image of the object taken with a scanner, saved as a 1200-dpi jpg image  
23 file, using AutoCAD® software. Furthermore, the length of at least one element of the object  
24 is identified and measured with the caliper. The image is inserted into AutoCAD® and scaled  
25 to the correct size using the reference lengths. Then, specific points are selected on the  
26 perimeter of the object image until all the points that enclose the area have been selected  
27 and a polyline is drawn through them. Finally, the software's Area tool calculates the area of  
28 the space enclosed by the sequence of specified points. This procedure is performed 30  
29 times to obtain the mean and standard deviation values.

### 3. Results and discussion

#### 3.1. Dependence of the SPP arrangement on thermomechanical processing

48 The grains morphology of the alloys after the rolling process, characterized with polarized  
49 light microscopy, is shown in Fig. 1. The results denote that a homogeneous microstructure,  
50 such as that of recrystallized grains, was formed by the additional cold rolling and annealing  
51 after a hot rolling of the  $\beta$ -quenched specimen. A banded structure along RD was not  
52 observed. Also, it is worth noting that is not evident a change in grain size by Ta addition.  
53 The average grain size was 4.1, 4.8 and 4.4  $\mu m$  in Zr-1.05Nb, Zr-0.85Nb-0.20Ta and Zr-  
54 0.85Nb-0.40Ta specimens, respectively, which are consistent with similar values reported in  
55 reference [22] and for commercial M5® alloy [23].



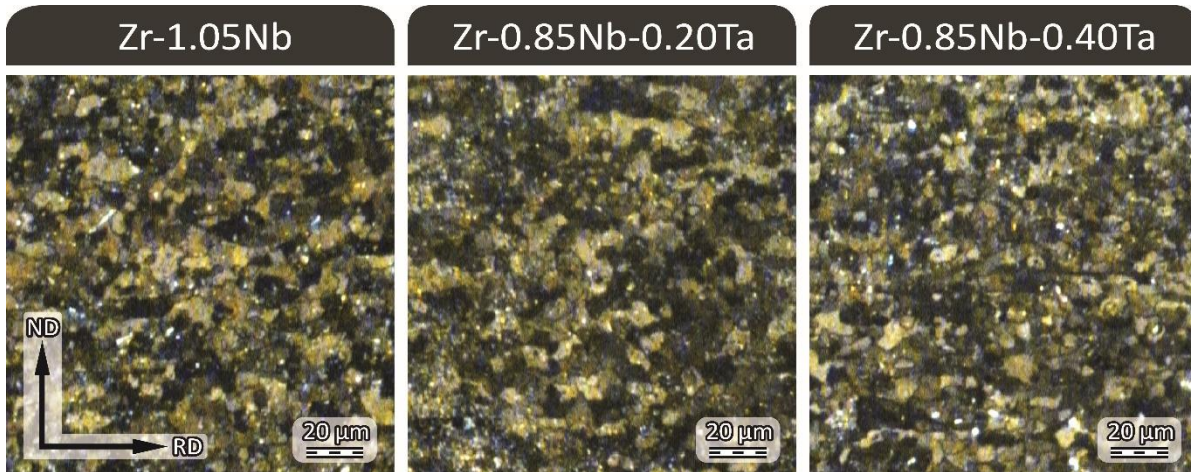
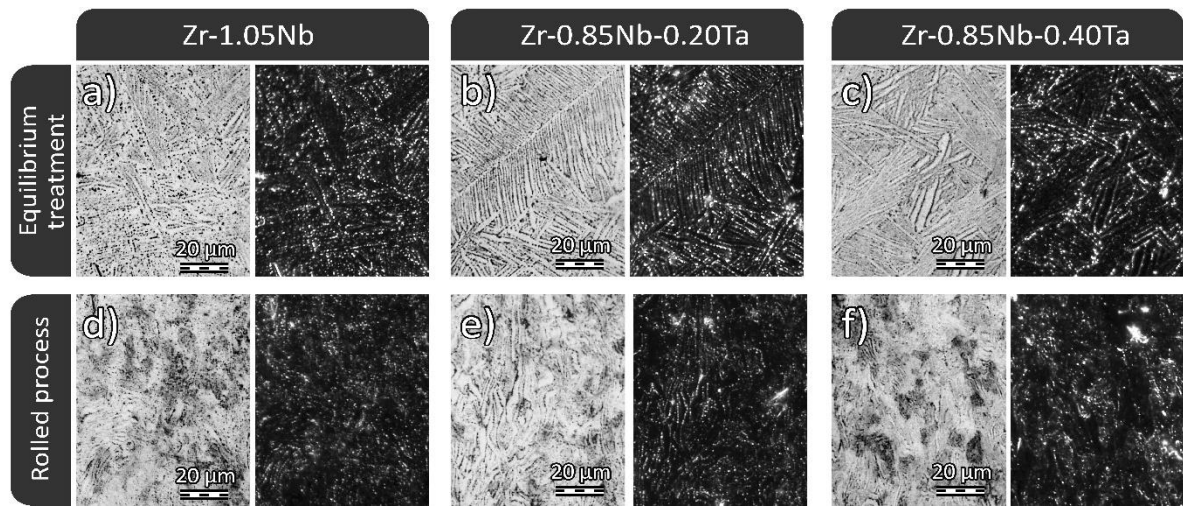


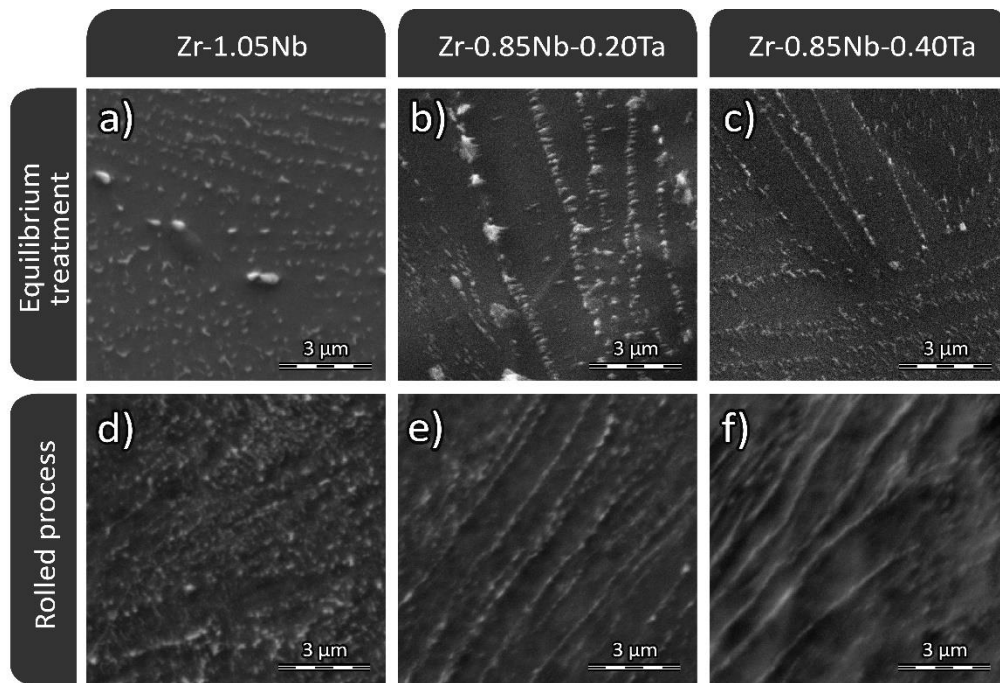
Fig. 1. Polarized optical micrographs of Ta-modified Zr-1Nb alloys after the rolling process.

The SPP's arrangement in the alloys, after the thermal treatments of  $\beta$  quenching and subsequent equilibrium annealing, are showed in the optical and SEM micrographs of Figs. 2a-c and 3a-c, respectively. As discussed in reference [15], in all three alloys, the SPP's arrangement agrees with the well-known precipitation mechanism of Zr-Nb alloys water-quenched from 1000 °C, or more, and annealed for a long time between 500 and 600 °C [[24],[25],[26]]. Precipitation occurs initially with a  $\beta$  phase nucleated as filaments at the needle boundaries and twin interfaces produced during the martensitic transformation  $\beta \rightarrow \alpha'$ , the  $\beta$  nuclei having the same composition as their parent  $\alpha'$  martensitic phase. On extended ageing, a subsequent Nb enrichment of these nuclei occurs until the filaments reach the monotectoid composition  $\beta_m$ , a further Nb enrichment starts the decomposition of the filaments with two-components, one is a Nb-rich precipitate and the other corresponds to the  $\alpha_{Zr}$  phase. The preferential precipitation at the needle boundaries can be seen from the bright field–dark field combination in the optical image of the same zone. The formation of a finer arrangement of smaller precipitates within the needles and along straight narrow bands marking the twin-boundary precipitation is shown in the SEM images. The platelike morphology of the SPP's, instead of the spherical one, represents the equilibrium form of non-coherent particles of  $\beta_{Nb}$  phase (its chemical composition and mass fraction is given in Table 1 for each alloy).

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**Fig. 2.** Optical micrographs of alloys with equilibrium treatment and rolling process. At right are the dark fields in which the precipitates are highlighted.



**Fig. 3.** SEM micrographs of alloys with the equilibrium treatment and the rolling process.

Figs. 2(d), (e) and (f) show the optical microstructures after the rolling process of the alloys in relation to its Ta content. The SPP's are arranged along wandering narrow bands that go through several recrystallized grains. When looking at this arrangement in more detail, as in the SEM micrographs in Figs. 3(d), (e) and (f), it is seen that a band line may have separate

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4 small precipitates if the alloy does not contain Ta or large elongated precipitates if it does.  
5 These results allow us to assume that filaments of  $\beta$  phase arranged in narrow bands are  
6 formed in the  $\beta$ -quenched martensite structure during preheating for 30 min at 570 °C.  
7 Subsequent hot and cold rolling processes make these bands wavy and wandering. Cold  
8 deformation adds the elastic energy necessary for the recrystallization process during  
9 intermediate annealing, the decomposition of  $\beta$  phase filaments also start during those  
10 annealing. The following section shows the results of the DSC measurements used to  
11 estimate the mass fraction of  $\beta_{Nb}$  phase precipitated during that  $\beta$  phase decomposition.  
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### 17 3.2. Beta niobium to beta zirconium transformation studies.

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19 At the end of thermal treatments of  $\beta$  quenching and subsequent equilibrium annealing,  
20 each alloy is studied by calorimetry. Each test consists of a heating/cooling cycle in the range  
21 450-1000 °C.  
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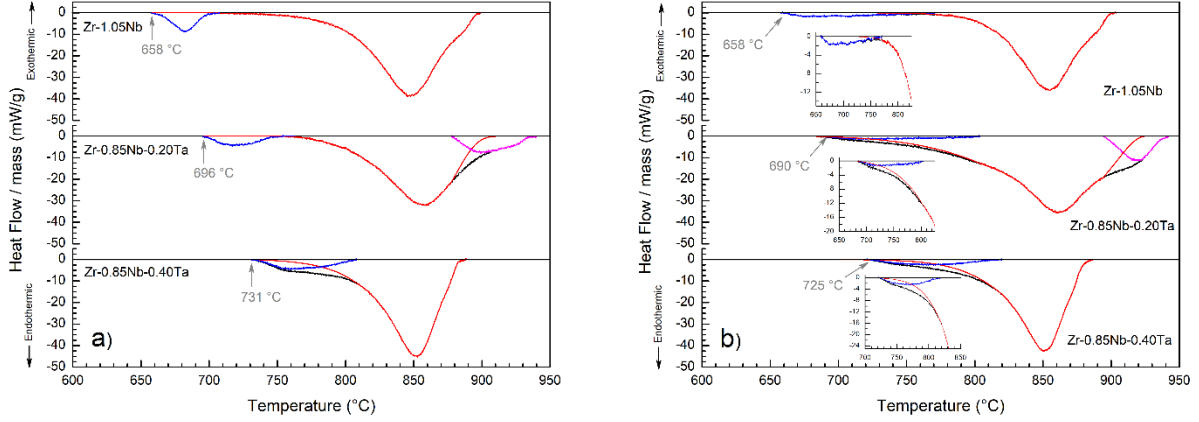
23 Fig. 4(a) represents a portion of the calorimetry thermogram obtained on heating. To  
24 enhance clarity, we have sketched only the transformation profile zone using the base-line  
25 subtracted DSC profiles. Two endothermic peaks are present: the first one corresponds to  
26 the dissolution of the  $\beta_{Nb}$  (into  $\beta_{Zr}$ ), and the second one corresponds to the overall allotropic  
27 transformation  $\alpha_{Zr} \rightarrow \beta_{Zr}$ . On cooling, not shown here, the rate of temperature change  
28 prevents the formation of the  $\beta_{Nb}$  precipitate and, therefore, only the peak corresponding  
29 to the allotropic transformation  $\beta_{Zr} \rightarrow \alpha_{Zr}$  is observed. Although the ending temperature  
30 for the allotropic transformation during cooling is imprecise, it is close to the starting  
31 temperature of the dissolution of the  $\beta_{Nb}$  precipitate. Therefore, this evidence implies a  
32 monotectoid transformation. The monotectoid temperature of the alloy is indicated by an  
33 arrow in each thermogram of the Fig. 4(a) and reflects an increase as the content of Ta  
34 increases.  
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41 The enthalpy change associate with the dissolution of the  $\beta_{Nb}$  precipitates can be estimated  
42 from the area under the peak observed in the plot of the heat flux per gram,  $(\dot{Q}/m_s)$ , of the  
43 alloy sample along the range temperature where this reaction occurs. To first order, this  
44 enthalpy change is proportional to the SPP's mass fraction  $C_{\beta_{Nb}}$  in such a way that [27]  
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$$48 \int \frac{\dot{Q}}{m_s} dT = v_T \times \frac{\Delta H}{m_s} = v_T \times \frac{\Delta H_{\beta_{Nb} \rightarrow \beta_{Zr}} m_{\beta_{Nb}}}{m_s} = v_T \times C_{\beta_{Nb}} \times \frac{\Delta H_{\beta_{Nb} \rightarrow \beta_{Zr}}^{molar}}{M_w} \quad (1)$$

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51 where  $\Delta H_{\beta_{Nb} \rightarrow \beta_{Zr}}$  and  $\Delta H_{\beta_{Nb} \rightarrow \beta_{Zr}}^{molar}$  are the mass and molar enthalpy change for the cited  
52 reaction, respectively;  $M_w$  is the molecular weight of the  $\beta_{Nb}$  precipitate and  $v_T$  is the heat  
53 rate. Table 2 shows the experimental values obtained for both definitions of the dissolution  
54 enthalpy, mass fraction and chemical composition of beta precipitates are those from Table  
55 1, the estimate errors come from propagation in eq. (1) including the error of the area under  
56 the peak. The deconvolution of the different enthalpic peaks is also shown in the Fig 4(a). A  
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third net reaction appears at the end of the allotropic transformation in the case of the alloy containing 0.2Ta, this phase transformation was not identified.



**Fig. 4.** Baseline-corrected DSC curves obtained on heating of the alloys after a) equilibrium annealing and b) rolling process. SPP's dissolution peak (blue line); allotropic transformation (red line), unidentified third phase transformation (magenta line).

**Table 2.** The  $\beta_{Nb/Ta} \rightarrow \beta_{Zr}$  transformation enthalpy obtained with calorimetric thermograms of alloy samples in thermodynamic equilibrium at 570 °C.

Alloy	$\Delta H/m_s$ (J/g)	$\Delta H_{\beta_{Nb} \rightarrow \beta_{Zr}}$ (J/g)	$M_w$ (g)	$\Delta H_{\beta_{Nb} \rightarrow \beta_{Zr}}^{molar}$ (kJ/mol)	$\Delta H_{\beta_{Nb/Ta}}^f$ (kJ/mol)
Zr-1.05Nb	$2.05 \pm 0.01$	$354 \pm 57$	92.61	$33 \pm 5$	$30.0 \pm 0.2$
Zr-0.85Nb-0.20Ta	$1.66 \pm 0.12$	$368 \pm 100$	103.35	$38 \pm 10$	$32.6 \pm 1.0$
Zr-0.85Nb-0.40Ta	$2.42 \pm 0.14$	$526 \pm 145$	109.2345	$57 \pm 16$	$34.1 \pm 1.2$

The pioneering work of Toffolon et al. [28] reports a two times higher estimate for  $\Delta H_{\beta_{Nb} \rightarrow \beta_{Zr}}$  in the Zr-1Nb alloy, in part the difference may be due to the fact that the  $C_{\beta_{Nb}}$  is estimated from the nominal alloy composition and the Zr-Nb phase diagram. However, this discrepancy is not enough to justify the value obtained in the present work, particularly if we try to justify the effect of Ta on the enthalpy of dissolution of the  $\beta_{Nb}$  precipitates. According to the literature [29],[30], the enthalpy of formation of a precipitate of composition  $M_m A_a B_b$  (where  $M$  is the majority component) from a solid solution formed by elements  $M$ ,  $A$  and  $B$ , can be estimated as follows,

$$(C_A^e)^a (C_B^e)^b = \exp\left(-\frac{\Delta H_{M_m A_a B_b}}{R T}\right) \quad (2)$$

where  $C_A^e$  and  $C_B^e$  are the concentrations of the  $A$  and  $B$  elements at their solubility limit in solvent  $M$  at temperature  $T$ . Using the data from Table 1 and eq. (2) we have estimated the enthalpy of formation of the  $\beta_{Nb/Ta}$  phase in our alloy samples, the results are shown in Table 2 as  $\Delta H_{\beta_{Nb/Ta}}^f$ . The uncertainty of the estimates was calculated by propagating errors in the eq. (2) and considering the various phase chemical compositions of Table 1. As can be seen, both our measurements and the semi-empirical thermodynamic estimates predict an increase in  $\Delta H_{\beta_{Nb/Ta}}^f$  with the addition of Ta. This also agrees with the strong increase in the critical solution point of  $\beta$  phase observed in the binary systems Zr-Nb [31] and Zr-Ta [32]. It also appears that the values of our measurements are closer than those of Toffolon et al. to the thermodynamic predictions.

From the above results, calorimetry appears to be a powerful tool to follow the progress of phase transformations in the SPP's during the rolling process, even for low volume fractions of them. Fig. 4(b) shows the calorimetry thermogram obtained on heating the rolled alloys. The graph inserts allow to see that the temperature range where the dissolution of the second phase particles is estimated to occur is larger than in the case of alloys in thermodynamic equilibrium at 570 °C. This can be explained by following the findings and conclusions of Toffolon et al. in reference [33], if some amount of  $\beta_{Zr}$  phase is present in the set of SPP's then the allotropic transformation start temperature is over evaluated and then the peaks deconvolution returns a transformation peak of  $\beta_{Nb}$  phase which temperature range is larger than the real one. Therefore, the mass fraction of  $\beta_{Nb}$  particles estimated with the calorimetric method will be overvalued. Using eq. (1), the mass fraction of  $\beta_{Nb}$  particles in the rolled alloys,  $C_{\beta_{Nb}}^{rolled}$ , was calculated as,

$$C_{\beta_{Nb}}^{rolled} = \left\{ \left[ \frac{\Delta H}{m_s} \right]^{rolled} / \left[ \frac{\Delta H}{m_s} \right]^{eq} \right\} C_{\beta_{Nb}}^{eq} \quad (3)$$

where the supra-index "eq" indicates the values of the variables for the alloys in thermodynamic equilibrium. Table 3 shows the measured values of  $\left[ \frac{\Delta H}{m_s} \right]^{rolled}$  and those calculated for  $C_{\beta_{Nb}}^{rolled}$  using the data from Table 2, the uncertainties in the latter were calculated by propagating errors in eq. (3). As expected, due to the lower mobility of Ta and higher enthalpy of formation of the  $\beta_{Nb/Ta}$  phase (taken as the quantities  $\Delta H_{\beta_{Nb} \rightarrow \beta_{Zr}}^{molar}$  in Table 2), the mass fraction  $C_{\beta_{Nb}}^{rolled}$  is lower in alloys containing Ta.

**Table 3.** Mass fraction of  $\beta_{Nb/Ta}$  phase from calorimetric thermograms of rolled alloys.

Alloy	$\left[\frac{\Delta H}{m_s}\right]^{rolled}$	$C_{\beta_{Nb}}^{rolled}$
	(J/g)	
Zr-1.05Nb	$1.34 \pm 0.17$	$0.38 \pm 0.11$
Zr-0.85Nb-0.20Ta	$1.22 \pm 0.18$	$0.33 \pm 0.19$
Zr-0.85Nb-0.40Ta	$1.73 \pm 0.12$	$0.33 \pm 0.11$

### 3.3. Hydrogen solubility

The results of the hydrogen solubility in alloys with the equilibrium annealing are summarized in Table 4. If the final hydrogen concentration of the alloy samples is determined by adding the initial hydrogen concentration of the zirconium sponge ( $\approx 45$  ppm) to that incorporated by gaseous charging, the TSSD(PT) value of Zr-1.08Nb alloy agrees very well with that reported by Khatamian et al. [34] for Zr-1Nb alloy aged at 500 °C for 1000 h. The general conclusion can be drawn that tantalum addition to Zr-1Nb alloy has a small influence. The trend of the effect with increasing Ta content shows a decrease in hydrogen solubility at low content, but then reverses this trend and increases it at higher Ta content. The observed hysteresis between cool-down and heat-up TSS of Zr-1.08Nb agrees with that reported by Parodi et al.[35] in Zr-2.5Nb aged for 168 h at 500 °C. There is no noticeable difference in this hysteresis with the addition of Ta, the hysteresis is associated with the mechanical properties of the  $\alpha_{Zr}$  phase [36] and they are expected to change little due to the small amount of Ta atoms added (see Table 1).

**Table 4.** The DSC hydride dissolution and precipitation data of alloy samples in thermodynamic equilibrium at 570 °C.

Alloy	Added hydrogen (ppm)	TSSD (°C)		TSSP (°C) $T_{max} = 500^\circ\text{C}$ Hold time = 10 min		$\Delta T$ (°C)	
		PT	MST	PT	MST	PT	MST
Zr-1.05Nb	198.0 $\pm$ 5.7	414.3 $\pm$ 0.6	432.9 $\pm$ 1.2	345.6 $\pm$ 0.5	371.9 $\pm$ 0.6	68.7 $\pm$ 1.1	61.0 $\pm$ 1.8
Zr-0.85Nb-0.20Ta	202.0 $\pm$ 8.2	420.5 $\pm$ 0.6	439.7 $\pm$ 0.7	347.1 $\pm$ 1.2	375.9 $\pm$ 0.4	73.4 $\pm$ 1.8	63.8 $\pm$ 1.1
Zr-0.85Nb-0.40Ta	198.0 $\pm$ 9.8	412.6 $\pm$ 0.9	429.5 $\pm$ 0.8	339.1 $\pm$ 1.4	376.2 $\pm$ 0.7	73.5 $\pm$ 2.3	53.3 $\pm$ 1.5

Khatamian et al.[37] has discussed the correlation between the TSSD curve for hydrogen in unalloyed Zr measured by Kearns using hydrogen partitioning in diffusion couples [38] with

that obtained through DSC measurements. Their conclusion was that, within the experimental errors, both correlate well if the peak temperature in DSC measurements is used. Table 5 shows the hydrogen concentration in unalloyed Zr,  $C_H^{\alpha_{Zr}}$ , calculated from the Kearns's curve using the TSSD(PT) measurements in our equilibrium aged alloys, as expected, these are lower than the total hydrogen content of them. The reason for this is the presence of  $\beta_{Nb}$  phase and the partitioning of hydrogen between the  $\beta_{Nb}$  and the  $\alpha_{Zr}$ . If we assume that the hydrogen concentration in the  $\alpha_{Zr}$  phase of our alloys is similar to that of unalloyed Zr (this is a valid assumption because the concentrations of Nb and Ta of this phase are low) and since we know its mass fraction (see Table 1), then we can estimate the hydrogen concentration in the  $\beta_{Nb}$  phase,  $C_H^{\beta_{Nb}}$ , through the partition of the total hydrogen,  $C_H^{total}$ , between both phases:

$$C_H^{\beta_{Nb}} = (C_H^{total} - (1 - C_{\beta_{Nb}}) C_H^{\alpha_{Zr}}) / C_{\beta_{Nb}}$$

Given the small value of the mass fraction of the  $\beta_{Nb}$  phase,  $C_{\beta_{Nb}}$ , both  $C_H^{\beta_{Nb}}$  and the partition ratio,  $\rho_H$ , have high values, as shown in Table 5. The uncertainty in  $C_{\beta_{Nb}}$  spreads giving rise to strong errors for the estimations of both,  $C_H^{\beta_{Nb}}$  and  $\rho_H$ , however, the change in chemical composition in the  $\beta_{Nb}$  phase seems to have an influence on the hydrogen partition with the  $\alpha_{Zr}$  phase.

The thermodynamic amounts of hydrogen dissolved in Nb-rich ternary alloys,  $Nb_{1-x-y}Zr_xTa_y$  in solid solution (bcc), are of interest because they provide information on the hydrogen absorption characteristics of aged Zr-1Nb alloys that contain a small amount of Ta. Following the thermodynamic model proposed by Ramaprabhu [39], the enthalpy of formation of the interstitial hydrogen in the Nb bcc lattice is linearly modified with the concentration of the substitutional atoms

$$(\Delta H_H^0)_{\beta_{Nb}} = (\Delta H_H^0)_{Nb} + \omega_{H \leftrightarrow Zr}^p C_{Zr}^{\beta} + \omega_{H \leftrightarrow Ta}^p C_{Ta}^{\beta} \quad (4)$$

where  $\omega_{H \leftrightarrow Zr}$  and  $\omega_{H \leftrightarrow Ta}$  are the interaction energies between hydrogen and the substitutional atoms Zr and Ta, respectively. The power "p" is the number of neighboring lattice sites around a given interstice. The entropy of formation of the interstitial in the alloy is considered like that of pure Nb,  $(\Delta S_H^0)_{alloy} \cong (\Delta S_H^0)_{Nb}$ . Thus, considering the configuration entropy, the equilibrium hydrogen concentration in the alloy is obtained as,

$$C_H^{\beta_{Nb}} = \exp \left[ -\frac{(\Delta G_H^0)_{\beta_{Nb}}}{R T} \right] = \exp \left[ -\frac{(\Delta H_H^0)_{\beta_{Nb}} - T(\Delta S_H^0)_{\beta_{Nb}}}{R T} \right]$$

$$C_H^{\beta_{Nb}} = \exp \left[ -\frac{(\Delta H_H^0)_{Nb} - T(\Delta S_H^0)_{Nb} + \omega_{H \leftrightarrow Zr}^p C_{Zr}^{\beta} + \omega_{H \leftrightarrow Ta}^p C_{Ta}^{\beta}}{R T} \right]$$

$$C_H^{\beta Nb} = C_H^{Nb} \exp \left[ -\frac{\omega_{H \leftrightarrow Zr}^p C_{Zr}^{\beta} + \omega_{H \leftrightarrow Ta}^p C_{Ta}^{\beta}}{R T} \right] \quad (5)$$

Table 5 shows the hydrogen concentration in pure Nb at the hydride dissolution temperature in our alloy samples as measured, at 760 Torr, from Veleckis et al [40]. The ratio between the hydrogen concentrations in the  $\beta_{Nb}$  phase and pure Nb, based on the values in Table 5, passes through a minimum for the Zr-0.85Nb-0.20Ta alloy. But the concentrations  $C_{Zr}^{\beta}$  and  $C_{Ta}^{\beta}$  decrease and increase monotonically following the same sequence in the alloys (see Table 1). Therefore, the thermodynamic model thus proposed (eq. (5)) would not account for this behavior.

**Table 5.** The hydrogen partition between phases of alloy samples in thermodynamic equilibrium at 570 °C.

Alloy	TSSD(TP) (°C)	$C_H^{\alpha Zr}$ (ppm)	$C_H^{\beta Nb}$ (ppm)	$\rho_H$	$C_H^{Nb}$ [40] (ppm)
Zr-1.05Nb	414.3±0.6	212.9±1.2	(5.6±1.2) 10 <sup>3</sup>	26.2±5.7	7232
Zr-0.85Nb-0.20Ta	420.5±0.6	225.7±1.3	(4.3±1.6) 10 <sup>3</sup>	19.1±7.0	7165
Zr-0.85Nb-0.40Ta	412.6±0.9	209.4±1.8	(7.7±1.6) 10 <sup>3</sup>	36.9±8.1	7250

The thermodynamic model is assuming a dilute substitutional alloy system, where the probability of finding the Zr and Ta atoms forming a complex of first neighbors is unlikely. But the higher the concentration of substitutional atoms, the higher that probability. In general, this probability is usually modeled by the product of the concentrations, so we propose to add in eq. (5) an additional term that now includes the interaction energy of hydrogen with the complex,  $\omega_{H \leftrightarrow Zr \cup Ta}$ , that is,

$$C_H^{\beta Nb} = C_H^{Nb} \exp \left[ -\frac{\omega_{H \leftrightarrow Zr}^p C_{Zr}^{\beta} + \omega_{H \leftrightarrow Ta}^p C_{Ta}^{\beta} + \omega_{H \leftrightarrow Zr \cup Ta}^{2p} C_{Zr}^{\beta} C_{Ta}^{\beta}}{R T} \right] \quad (6)$$

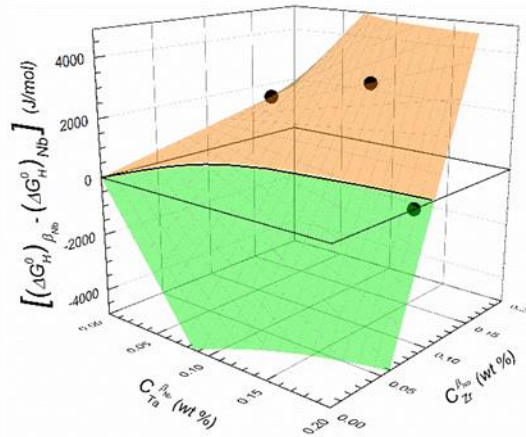
To estimate the characteristics of the pair interactions in eq. (6) we use the data in Table 5 and obtain the following results

$$\omega_{H \leftrightarrow Zr}^p \cong 8.6 \text{ kJ/mol} \parallel \omega_{H \leftrightarrow Ta}^p \cong -56.8 \text{ kJ/mol} \parallel \omega_{H \leftrightarrow Zr \cup Ta}^{2p} \cong 507.5 \text{ kJ/mol} \quad (7)$$

whose consequence on the solubility of hydrogen in the alloy rich in Nb is better displayed in the Fig 5. The orange-colored surface of the plot indicates the chemical composition of the  $\beta_{Nb}$  phase where the free energy of formation of interstitial hydrogen in the alloy is greater than in pure Nb, that is, the solubility of H decreases in the alloy with respect to pure Nb. The opposite effect occurs on the green colored surface. This result is in agreement with the findings known in the literature [[41],[42]], solute atoms with a size smaller than the solvent, for example V (1.34 Å) and Mo (1.36 Å) in Nb (1.43 Å), increase the terminal solubility



of hydrogen (TSH) rapidly as the solute concentration increases, while solutes with slightly larger sizes, for example Ta (1.44 Å) in Nb, also increase TSH but less rapidly. The cases of large solute sizes, such as Zr (1.54 Å) in Nb, were not specifically studied for hydrogen. However, Szkopiak et al.[43] have studied, by the internal friction technique, the solute-interstitial (s-i) interactions in thirteen niobium-1 at% substitutional alloys containing oxygen and nitrogen as interstitial solutes. They find that the lowest s-i interaction occurs between Zr or Hf with both interstitial atoms. In the ternary bcc alloys, the effect of the pairing of substitute solute atoms on the s-i interaction was also studied with the same experimental technique [44]. Our results show that given a concentration of Zr in Nb, the addition of isolated Ta atoms should increase the solubility of hydrogen, but the pairing of a few Zr and Ta atoms retards the effect.



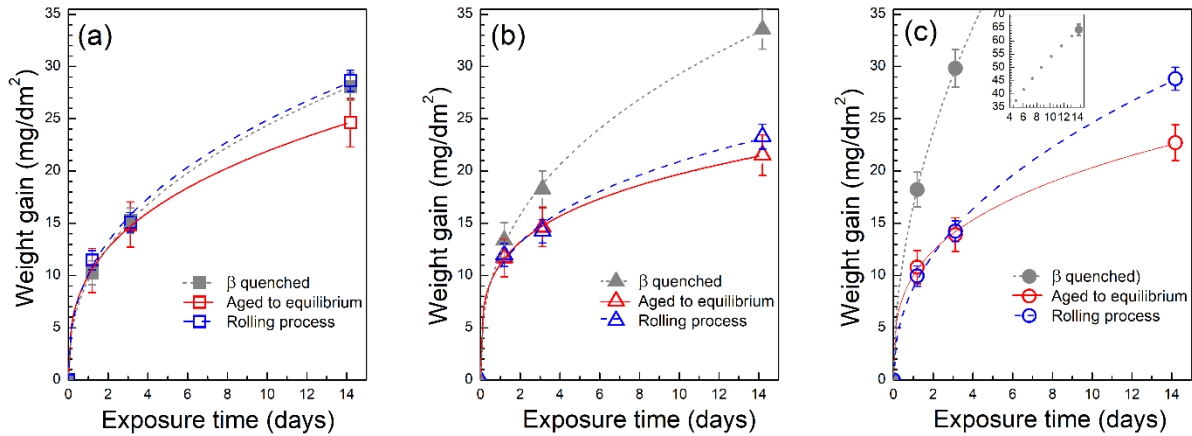
**Fig. 5.** Free energy difference for interstitial hydrogen formation in  $\beta_{Nb}$  phase and in pure Nb as a function of the concentration of Zr,  $C_{Zr}^{\beta}$ , and Ta,  $C_{Ta}^{\beta}$ . The surface is calculated with the values of interaction energies in eq. (7) while the symbols indicate the isolated values calculate with  $C_{Zr}^{\beta}$  and  $C_{Ta}^{\beta}$  given in Table 1.

### 3.4. Corrosion characteristics

The weight gains for Zircaloy-4 control coupons (T, M and B) under steam corrosion at 400 °C and 10 MPa for 14 days were in the range  $25.0 \pm 1.8 \text{ mg/dm}^2$  and the oxide visual appearance was uniform, black, lustrous and adherent which shows that the test achieved the establish requirements for Zircaloys alloys in the standard test [21] and was valid. Furthermore, the oxide film on all the Zr-Nb-Ta alloys studied resembles that of the Zircaloy-4 control coupons, except for a dark gray color instead of black.

Fig. 6 shows the effect of thermo-mechanical processing and tantalum addition on the corrosion behavior of Zr-1Nb alloys. Regardless of the thermomechanical processes used, the weight gain of the alloys with equilibrium annealing or rolling process are within the

range ( $18.3 \leq \Delta w \leq 38.1$ )  $mg/dm^2$  required for the Zircaloy alloys [21]. The corrosion test carried out in this investigation is mainly used as zirconium alloys' acceptance test according to ASTM G2/G2M-06 standard, but in such a role, establishes a tolerance and, therefore, has been widely used in new alloys' development. In this sense, it seems appropriate to provide an in-depth analysis of the results obtained.



**Fig. 6.** The effects of thermo-mechanical processing and tantalum addition on the corrosion behavior of Zr-1Nb alloys corroded at 400 °C steam for 14 days: a) Zr-1.05Nb; b) Zr-0.85Nb-0.20Ta and c) Zr-0.85Nb-0.40Ta

The analysis starts observing that the weight gain of the water-quenched specimens is higher than those that have reached the state of thermodynamic equilibrium at 570 °C. This behavior was studied by Jeong et al. [[2],[3]] in Zr-Nb alloys. The first study [2] showed that if the Nb content in the alloy is greater than 0.6 wt%, the resistance to corrosion of the water-quenched sample decreases as the amount of Nb in the supersaturated state increases above that Nb concentration limit. However, for the alloy without Ta, the decrease in corrosion resistance in our experiment is less than in Jeong's. The difference is largely due to the amount of Nb in the supersaturated state in both water-quenched samples. Our alloy sample was extracted from a button that was water-quenched from beta phase, while Jeong's samples are pieces of water-quenched sheet metal, this makes for a much lower cooling rate for our sample. When the cooling rate is slowed down to about 100°C/s, the filaments of the  $\beta$  phase nucleate early at the needle boundaries and twin interfaces produced during the martensitic transformation  $\beta \rightarrow \alpha'$  and part of Nb atoms segregate within them [[45],[46]]. Accordingly, the Nb in the supersaturated matrix is lower than that obtained at higher cooling rate and the corrosion resistance increases [2]. As the cooling rate was the same for buttons with and without Ta, the same loss of Nb is expected in the supersaturated matrix of samples with Ta added, but Nb diffuses faster than Ta in Zr [47], therefore, Ta should remain in the supersaturated state in the matrix. In addition, if Ta in solution is as detrimental as Nb for corrosion resistance of zirconium, the corrosion behavior

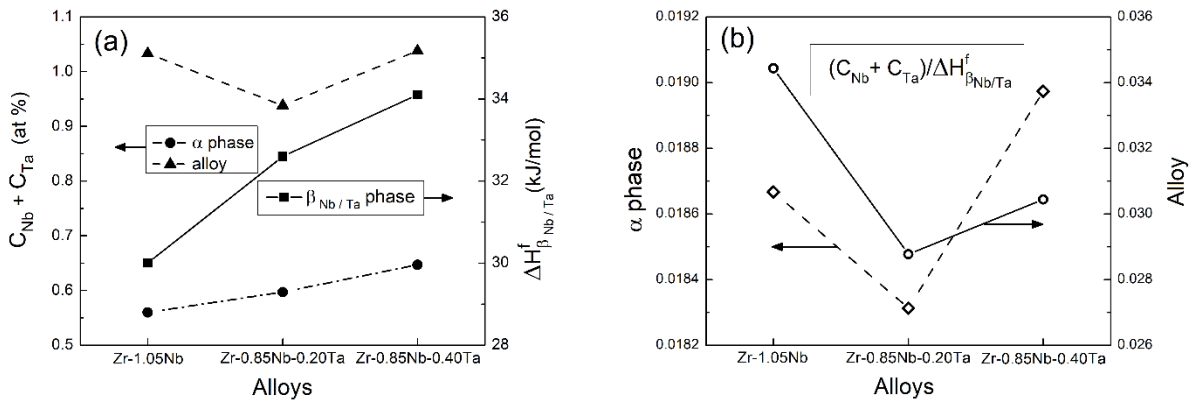
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4 shown in Fig. 6 by the water-quenched samples containing Ta is consistent with the  
5 hypotheses stated.  
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7 If the previous analysis is applied to the samples in thermodynamic equilibrium at 570 °C,  
8 which its Nb plus Ta content in the  $\alpha_{Zr}$  matrix is taken from Table 1, it predicts a monotonous  
9 deterioration in corrosion resistance with increasing content of Ta added to the alloy, but  
10 this is not the case, both alloys containing Ta show a tendency to increase corrosion  
11 resistance in 14 days with respect to which does not contain it and the highest resistance  
12 would be for the alloy Zr-0.85Nb-0.20Ta. In reference [3] Jeong et al. show the results of the  
13 corrosion resistance of the water-quenched Zr-Nb samples of reference [2] which were  
14 annealed for 500 hs at 570 °C to obtain the  $\alpha_{Zr} + \beta_{Nb}$  phase equilibrium. At high Nb contents  
15 of 1.0-5.0 wt%, the formation of the  $\beta_{Nb}$  phase resulted in the reduction of the Nb  
16 concentration in the  $\alpha_{Zr}$  matrix and the corrosion resistance was improved, but the  
17 improvement remained nearly steady with the  $\beta_{Nb}$  volume fraction. Therefore, our results  
18 with the addition of Ta to the  $\beta_{Nb}$  phase imply that its chemical composition is important for  
19 the corrosive behavior of the alloy. In the following lines we will try to understand this  
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23 In the studies by Moorehead et al. [5] on the role of Nb in the oxidation kinetics of Zr-Nb  
24 alloys, they conclude that the parabolicity of this kinetics is governed, in part, by the space  
25 charge in the oxide layer. This space charge can be compensated by solute aliovalent Nb ions  
26 spread out in the oxide. The source of these ions are the various phases of the alloy, and the  
27 atomic order of those phases seems to have effect in their dissolution kinetics, for example,  
28 these researchers demonstrated that  $\beta_{Zr}$  precipitates dissolve faster than  $\beta_{Nb}$  precipitates  
29 therefore, the latter are more effective in retaining Nb and retarding the growth of the oxide  
30 layer. Since the corrosion tests for the different microstructures of the same alloy were  
31 carried out simultaneously in a single aqueous corrosive environment, the change in  
32 chemical activity of the various species due to the change in the thermodynamic equilibrium  
33 of the phases shows a magnitude able to produce a significant change in the corrosion  
34 behavior of this kind of alloy [48]. With these ideas in mind and thinking about the corrosion  
35 resistance in ours samples in thermodynamic equilibrium at 570 °C, we observe in Table 2  
36 that the estimated enthalpy of formation of the  $\beta_{Nb/Ta}$  phase,  $\Delta H_{\beta_{Nb/Ta}}^f$ , increases with the  
37 Ta added to the alloy, and since the alloy is biphasic, this means that the activity of Nb and  
38 Ta also increases in the  $\alpha$  matrix and therefore the dissolution kinetic of both phases  
39 decrease.  
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42 In summary, one can expect the growth of the oxide layer to be proportional to the atomic  
43 concentration of Nb plus Ta and inversely proportional to the chemical activity of these  
44 species. Fig. 7a shows the atomic concentrations of Nb and Ta in the alloy and in the  $\alpha$  phase  
45 for the three samples, and the enthalpy of formation of the  $\beta_{Nb/Ta}$  phase indicative of the  
46 chemical activity of the species in the equilibrium phases. Fig. 7b shows the ratio between  
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the sum of atomic concentrations and the formation enthalpy, that is, a predictor of the growth behavior of the oxide layer. Comparing the predictor and the experimental observations in Fig. 6, we see that the predictor calculated with the atomic concentrations of Nb plus Ta in the alloy correctly predicts them while the one calculated only with the sum of the concentrations in the matrix does not. The decomposition of  $\beta_{Zr}$  precipitates during the thermomechanical rolling and annealing process causes the chemical activity of the species to gradually increase towards the maximum value at the phase equilibrium, therefore, the growth rate of the oxide layer should be expected intermediate between that observed in the quenched sample and the sample in thermodynamic equilibrium, as can be seen in all the cases illustrated in Fig. 6.



**Fig. 7.** Predictor of the growth behavior of the oxide layer, at 400 °C steam for 14 days, in samples of Zr-1Nb alloys added with Ta and aged until equilibrium at 570 °C.

#### 4. Conclusions

The results of our research indicate that a small addition of Ta maintains the biphasic microstructure of Zr-1Nb nuclear alloys by increasing their monotectoid transformation temperature and their resistance to corrosion. Although a slight decrease in hydrogen solubility was observed at low Ta content.

Even though the corrosion resistance testing of the Ta-containing experimental alloys should continue to corroborate the outstanding trend observed at 14 days, it is worth further research of the Zr-1Nb-xTa alloy system as a candidate material for applications in the nuclear reactor core. Mainly when considering the higher thermal microstructural stability compared to Ta-free Zr-1Nb alloys. This could provide advantages such as a possible increase in operating temperature by tens of degrees or, similarly, seeking an acceleration in precipitation kinetics during thermomechanical processes.

## Acknowledgments

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P.A. Ferreiros: Conceptualization, Methodology, Investigation, Formal analysis, Validation, Writing - Original Draft, Visualization

E. Savoy Polack: Investigation, Data Curation

L. Lanzani: Resources, Methodology, Formal analysis, Writing - Original Draft

P.R. Alonso: Resources, Writing - Original Draft

D.P. Quiros: Resources, Investigation.

J.I. Mieza: Resources, Investigation, Data Curation

G.H. Rubiolo: Conceptualization, Methodology, Formal analysis, Writing - review & editing, Supervision, Project administration, Funding acquisition.